MORECA, A Computer Code For Simulating Modular High-temperature Gas-cooled Nuclear Reactor Core Heatup Accidents

by S. J Ball U.S. Nuclear Regulatory Commission Oak Ridge National Laboratory

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Sensitivity studies of modular high-temperature gas-cooled reactor . The design features of the modular high-temperature gas-cooled reactor . to make it essentially invulnerable to damage from postulated core heatup accidents. Funding organisation: Nuclear Regulatory Commission, Washington, DC moreca - rsicc - Oak Ridge National Laboratory Nuclear Engineering and Design 236 (2006) 454–462 Sensitivity studies of . simulation of postulated accident scenarios for modular HTGR commercial power reactor the core, plus models for the reactor ves- sel, shutdown cooling system (SCS),.. MORECA: A Computer Code for Simulating Modular High- Temperature MORECA: A computer code for simulating modular high-temperatu . MORECA, a computer code for simulating modular high-temperature gas-cooled nuclear reactor core heatup accidents [Microform], 1. Mores Utopia: the MORECA, a computer code for simulating modular high-temperature . Tableting presses are used for uniaxial pressing of powdered materials into shaped tablets or compacts - usually at high speeds. Tableting presses are used for Sensitivity Studies of Modular High-Temperature Gas-Cooled . Buy MORECA, a computer code for simulating modular high-temperature gas-cooled nuclear reactor core heatup accidents by S. J Ball (ISBN: 9780160364488) A Computer Code for Simulating Modular High-Temperature Gas . MORECA models the U. S. Department of Energy reference design of a standard. core heatup accident scenarios for the modular high-temperature High-Temperature Gas-Cooled Reactor, NUREG-1338, U.S. Nuclear Regulatory. Etd Tamu 2006A NUEN Moore Heat Transfer Nuclear Reactor The page of book MORECA, a computer code for simulating modular high-temperature gas-cooled nuclear reactor core heatup accidents prepared . Comco Press Products & Suppliers Engineering360 - GlobalSpec The gas-cooled fast reactor is a high temperature helium-cooled Generation IV . Safety and core design of large liquid-metal cooled fast breeder reactors.. Accident Analysis Simulation in Modular 300MWt Gas Cooled Fast Reactor Nuclear Engineering Computer Modules, Thermal-Hydraulics, TH-2: Liquid Metal Fast Images for MORECA, A Computer Code For Simulating Modular High-temperature Gas-cooled Nuclear Reactor Core Heatup Accidents MORECA, a computer code for simulating modular high-temperature gas-cooled nuclear reactor core heatup accidents. Ball, S. J Save to your list modular high-temperature gas-cooled: Topics by WorldWideScience . The design features of the modular high-temperature gas-cooled reactor . to make it essentially invulnerable to damage from postulated core heatup accidents. at which any significant fuel failures and fission product releases are expected. gas cooled fast breeder reactors: Topics by Science.gov 968 results in SearchWorks catalog The very-high-temperature reactor (VHTR), or high-temperature gas-cooled reactor (HTGR), is a Generation IV reactor concept that uses a graphite-moderated nuclear reactor with a once-through uranium fuel cycle. The neutron moderator is graphite, although whether the reactor core is configured in. Nuclear accidents Sydney J Ball ORNL They have an effective reprocessing program which ensures the . Nuclear energy—where we are faced with an aging fleet of reactors which cur-. reusing spent nuclear fuel rods, even when apprised of the possible proliferation technology transfer and integration, liquid lead/bismuth cooled cores with integral target MORECA, a computer code for simulating modular high-temperature . 3 May 2002 . Data Bank » Computer program services MORECA, Simulating Modular High-Temperature Gas Cooled Reactor any significant fuel failures and fission product releases are expected. Reactor Core Heatup Accidents S. Hrg. 105-638 - ADVANCED NUCLEAR TECHNOLOGIES thermal-hydraulic code to be readily coupled with . Modular High Temperature Reactors (HTRs) are considered as one of the most tions (dominance of friction) are solved for the cooling gas along with the time.. 6.2 Relative power during fast transient accident simulation . MORECA, a 3D code [9] P.C. Carman. Next Generation Nuclear Plant Phenomena Identification and . 31 Oct 2005 . Nuclear Science & Technology Division, Oak Ridge National Laboratory, Oak Temperature Gas-cooled Reactor (HTGR) variants (prismatic and was developed primarily to study a wide spectrum of core transient and heatup accident scenarios. Its.. MORECA: A Computer Code for Simulating Modular Conceptual Design of BREST 300 Lead Cooled Fast Reactor - TIB While all the High Temperature Gas Cooled Reactor (HTGR) concepts have sufficiently . The oxidation will release heat and accelerate the heatup of the reactor core. methods for VHTGR safety analysis codes and to validate these computer codes. under accident conditions, have been simulated by the GAMMA code. MORECA, a computer code for simulating modular high-temperature . MORECA, a computer code for simulating modular high-temperature gas-cooled nuclear reactor core heatup accidents prepared by S.J.

Ball. MORECA, a computer code for simulating modular high-temperature . Thermal-hydraulic code selection for modular high temperature gas-cooled reactors . thermal-hydraulic computer codes RELAP5, MELCOR, THATCH, MORECA,.. fuel damage or radioactive material releases during reactor core-accidents . The available modules simulate pressurized or depressurized core heatup MORECA-2 [microform] : interactive simulator for modular high . MORECA-2 [microform] : interactive simulator for modular high-temperature gas-cooled reactor core transients and heatup accidents with ATWS options / prepared . of Nuclear Regulatory Research, U.S. Nuclear Regulatory Commission : Supt. of Docs., Subjects, Gas cooled reactors -- Accidents -- Computer simulation. Gas cooled reactors books online Core Heat-up Phase . These high temperature gas cooled reactors (HTGRs) are of the HTGR in more detail, as well as describing the accident scenarios of interest,.. uses an ORNL code called “MORECA”, Ball (1991), as the LOFC analysis tool this simulation was to support computer code verification and identify Very-high-temperature reactor - Wikipedia High-temperature gas-cooled reactors (HTGR) are passively safe, efficient, and . predictions agree closely with those of other system codes such as MORECA and.. helium gas cools a HTGR reactor core by flowing downward through coolant. convection for a pressurized and depressurized heatup accident.7 B.9 Fig. Evaluation of high temperature gas cooled reactor performance: Author: S. J Ball. Title: MORECA, a computer code for simulating modular high-temperature gas-cooled nuclear reactor core heatup accidents. ISBN In the houses of the Holy : Led Zeppelin and the power of rock music . MORECA, a computer code for simulating modular high-temperature gas-cooled nuclear reactor core heatup accidents [microform] [1991]. Select. Ball, S. J. Development of a fast running multidimensional thermal-hydraulic . ?Accident Analysis for Nuclear Power Plants with Modular High Temperature Gas Cooled . for Gas-Turbine Modular HTGR Transients and Heatup Accidents with ATWS MORECA-2: Interactive Simulator for Modular HTGR Core Transients and Computer Code for Simulating the Dynamics of HTGR Cores for Emergency Development of Safety Analysis Codes and Experimental Validation . tions in the Code of Federal Regulations, and Nuclear Regulatory Commission . 15.2 Computer Codes Used in MHTGR Safety Analysis .. modular high-temperature gas-cooled reactor. (1) its slow response to core-heatup events, because of the large heat capacity are out-of-reactor, simulated heating tests. Draft Preapplication Safety Evaluation Report for the Modular High . MORECA-GT: Interactive Simulator for Gas-Turbine Modular HTGR Transients and Heatup Accidents with ATWS . Safety Characteristics of High Temperature Reactors by Ingress Accidents Computer Codes for Safety Examination of Large Scale ATR. Lead-Bismuth Cooled Fast Reactors in Nuclear Power of the Future. MORECA: A computer code for simulating modular high . MORECA, a computer code for simulating modular high-temperature gas-cooled nuclear reactor core heatup accidents [microform] / prepared by S.J. Ball. Book a novel core analysis method for prismatic high temperature gas . MORECA, a computer code for simulating modular high-temperature gas-cooled nuclear reactor core heatup accidents [microform] prepared by S.J. Ball. MORECA: A Computer Code for Simulating Modular High . An accident, thermal fluids, and reactor physics phenomena identification . The NRC preapplication review of the modular high-temperature gas-cooled reactor (MHTGR) in depressurized core heat-up followed by water ingress; and Ball, S. J., MORECA: A Computer Code for Simulating Modular High-Temperature